

Operation challengers for CFETR and suggestions

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- 1. Usual challengers**
- 2. Special challengers for CFETR**
- 3. Summary**

1. Usual challengers

Similar to ITER, CFETR will face some usual challengers, as briefly mentioned below

1.1 Disruptions

- * The disruption prediction and mitigation should be studied and developed in advance.**
- * ECRH/ECCD/LHCD for disruption avoidance should be studied and developed in advance.**

1.2 ELM mitigation

- * Install RMP coils for ELMs mitigation?**
- * Pellet pace-making?**
- * Studies are required to understand ELMs and their mitigation mechanisms.**

1.3 Compatibility of Core and Radiative Divertor

- * Predicted narrow power deposition on divertor plate between ELMs.**
- * requires plasma detachment from the divertor.**
- * Excessive core impurities due to seeding for radiative divertor.**

Possible measures:

- * Impurities may be limited by high power core electron heating.**

1.4 Other issues

- * H-mode power threshold?**
- * density limit/peaking?**
- * Vertical instability control**
- * ...**

2. Special challengers for CFETR

2.1 Design requirements and machine parameters

Requirements

Fusion power	50~ 200MW
Based on Standard H-mode	H=1
duty time	0.3 ~ 0.5;
Tritium self-sufficiency for DT fuel cycle	TBR \geq 1.2

Major parameters

B_{to}	5.3 / 4.5	T
I_p	12 / 10/ 07	MA
R_o	5.5	m
a	1.6	m
Elongation K	1.8	
Blanket thickness:	1.0	m

2.2 Estimations based on scaling laws

In order to carry out calculations, the following **assumptions** are made:

- 1) Same magnet technology as ITER's.
- 2) The normalized beta value is assumed to be $\beta_N \leq 2.5$.
- 3) H-mode factor $H_h=1.0$ (conventional H-mode).
- 4) The current drive efficiency is assumed to be

$$\gamma = I_{cd} * R * \langle n_e \rangle / P_{cd} = 0.2 * 10^{20} \text{ A}/(\text{m}^2\text{W})$$

I_{cd} is the driven current,

$\langle n_e \rangle$ the volume averaged electron density,

R the major radius,

P_{cd} the non-inductive current drive power.

5) Hydrogen plasma for energy confinement time and L-H transition power threshold.

6) The electron density is $\langle n_e \rangle = 0.8 n_{GW}$

Under above assumptions, some calculation results are given below.

2.2.1 $B_t = 5.3$ T, $I_p = 12.0$ MA

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

Toroidal field

$$B_t = 5.3 \text{ T}$$

Plasma current

$$I_p = 12.0 \text{ MA}$$

Energy confinement time

$$\tau_E = 2.47 \text{ s} \quad (\text{for } P_{\text{heating}} = 73 \text{ MW})$$

L-H transition power

$$P_{L-H} = 99.5 \text{ MW}$$

Safety factor

$$q_{95} = 2.89$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{bs} = 0.298$$

Total volt-seconds

$$\Phi_{\text{tot}} = 71 \text{ Wb} \quad (\Phi_{\text{inductive}} = 58 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = 545.8 \text{ s}$$

Electron density (vol. average)

$$\langle n_e \rangle = 1.194 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{cd} = 0.5$$

Non-inductive current drive power

$$P_{cd} = 197 \text{ MW}$$

comments: $q_{95} = 2.89 < 3$, $P_{cd} = 197$ MW for 545.8 s burning time, with assumed $B_t = 5.3$ T, $I_p = 12.0$ MA, $\beta_N = 2.5$ and $f_{cd} = 0.5$.

2.2.2 $B_t = 5.3$ T, $I_p = 10.0$ MA

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

toroidal field

$$B_t = 5.3 \text{ T}$$

Plasma current

$$I_p = 10.0 \text{ MA}$$

Energy confinement time

$$\tau_E = 1.94 \text{ s} \quad (\text{for } P_{\text{heating}} = 73 \text{ MW})$$

L-H transition power

$$P_{L-H} = 89 \text{ MW}$$

Safety factor

$$q_{95} = 3.47$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{bs} = 0.358$$

total volt-seconds

$$\Phi_{\text{tot}} = 71 \text{ Wb} \quad (\Phi_{\text{inductive}} = 48 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = 1609 \text{ s}$$

Electron density (vol. average)

$$\langle n_e \rangle = 0.995 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{cd} = 0.5$$

Non-inductive current drive power

$$P_{cd} = 136.8 \text{ MW}$$

comments: $P_{cd} = 136.8$ MW for 1609 s burning time, with assumed $B_t = 5.3$ T,

$I_p = 10.0$ MA, $\beta_N = 2.5$ and $f_{cd} = 0.5$.

2.2.3 $B_t = 4.5$ T, $I_p = 10.0$ MA

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

toroidal field

$$B_t = 4.5 \text{ T}$$

Plasma current

$$I_p = 10 \text{ MA}$$

Energy confinement time

$$\tau_E = 1.89 \text{ s} \quad (P_{\text{heating}} = 73 \text{ MW})$$

L-H transition power

$$P_{\text{L-H}} = 78 \text{ MW}$$

Safety factor

$$q_{95} = 2.946$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{\text{bs}} = 0.30$$

total volt-seconds

$$\Phi_{\text{tot}} = 71 \text{ Wb} \quad (\Phi_{\text{inductive}} = 48 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = 1165 \text{ s}$$

Electron density (vol. average)

$$\langle n_e \rangle = 0.995 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{\text{cd}} = 0.5$$

Non-inductive current drive power

$$P_{\text{cd}} = 136.8 \text{ MW}$$

comments: $P_{\text{cd}} = 136.8$ MW for 1165 s burning time, with assumed $B_t = 4.5$ T, $I_p = 10$ MA, $\beta_N = 2.5$ and $f_{\text{cd}} = 0.5$.

2.2.4 $B_t = 4.5 \text{ T}$, $I_p = 10.0 \text{ MA}$

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

toroidal field

$$B_t = 4.5 \text{ T}$$

Plasma current

$$I_p = 10 \text{ MA}$$

Energy confinement time

$$\tau_E = 1.89 \text{ s} \quad (P_{\text{heating}} = 73 \text{ MW})$$

L-H transition power

$$P_{\text{L-H}} = 78 \text{ MW}$$

Safety factor

$$q_{95} = 2.946$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{\text{bs}} = 0.30$$

total volt-seconds

$$\Phi_{\text{tot}} = 71 \text{ Wb} \quad (\Phi_{\text{inductive}} = 48 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = \text{infinity}$$

Electron density (vol. average)

$$\langle n_e \rangle = 0.995 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{\text{cd}} = 0.7$$

Non-inductive current drive power

$$P_{\text{cd}} = 191 \text{ MW}$$

comments: $P_{\text{cd}} = 191 \text{ MW}$ for steady operation, with assumed $B_t = 4.5 \text{ T}$, $I_p = 10 \text{ MA}$, $\beta_N = 2.5$, and $f_{\text{cd}} = 0.7$.

2.2.5 $B_t = 4.5$ T, $I_p = 7.0$ MA

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

toroidal field

$$B_t = 4.5 \text{ T}$$

Plasma current

$$I_p = 7.0 \text{ MA}$$

Energy confinement time

$$\tau_E = 1.17\text{s} \quad (P_{\text{heating}} = 73\text{MW})$$

L-H transition power

$$P_{\text{L-H}} = 63 \text{ MW}$$

Safety factor

$$q_{95} = 4.2$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{\text{bs}} = 0.43$$

total volt-seconds

$$\Phi_{\text{tot}} = 71\text{Wb} \quad (\Phi_{\text{inductive}} = 34 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = \text{infinity}$$

Electron density (vol. average)

$$\langle n_e \rangle = 0.696 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{\text{cd}} = 0.57$$

Non-inductive current drive power

$$P_{\text{cd}} = 76 \text{ MW}$$

comments: $P_{\text{cd}} = 76$ MW for steady operation, with assumed $B_t = 4.5\text{T}$, $I_p = 7.0$ MA, $\beta_N = 2.5$ and $f_{\text{cd}} = 0.57$.

2.2.6 $B_t = 4.0 \text{ T}$, $I_p = 7.0 \text{ MA}$

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

toroidal field

$$B_t = 4.0 \text{ T}$$

Plasma current

$$I_p = 7.0 \text{ MA}$$

Energy confinement time

$$\tau_E = 1.15 \text{ s} \quad (P_{\text{heating}} = 73 \text{ MW})$$

L-H transition power

$$P_{\text{L-H}} = 57 \text{ MW}$$

Safety factor

$$q_{95} = 3.74$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{\text{bs}} = 0.386$$

total volt-seconds

$$\Phi_{\text{tot}} = 71 \text{ Wb} \quad (\Phi_{\text{inductive}} = 34 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = \text{infinity}$$

Electron density (vol. average)

$$\langle n_e \rangle = 0.696 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{\text{cd}} = 0.62$$

Non-inductive current drive power

$$P_{\text{cd}} = 83 \text{ MW}$$

comments: $P_{\text{cd}} = 83 \text{ MW}$ for steady operation, with assumed $B_t = 4.0 \text{ T}$, $I_p = 7.0 \text{ MA}$, $\beta_N = 2.5$ and $f_{\text{cd}} = 0.62$.

2.2.7 $B_t = 3.5 \text{ T}$, $I_p = 7.0 \text{ MA}$

Major/minor radius

$$R/a = 5.5/1.6 \text{ m}$$

toroidal field

$$B_t = 3.5 \text{ T}$$

Plasma current

$$I_p = 7.0 \text{ MA}$$

Energy confinement time

$$\tau_E = 1.1\text{s} \quad (P_{\text{heating}} = 73\text{MW})$$

L-H transition power

$$P_{\text{L-H}} = 52 \text{ MW}$$

Safety factor

$$q_{95} = 3.27$$

Normalized beta value

$$\beta_N = 2.5$$

Bootstrap current fraction

$$f_{\text{bs}} = 0.34$$

total volt-seconds

$$\Phi_{\text{tot}} = 71\text{Wb} \quad (\Phi_{\text{inductive}} = 34 \text{ Wb})$$

Burning time

$$T_{\text{burn}} = \text{infinity}$$

Electron density (vol. average)

$$\langle n_e \rangle = 0.696 \quad (10^{20}/\text{m}^3)$$

Non-inductive current drive fraction

$$f_{\text{cd}} = 0.67$$

Non-inductive current drive power

$$P_{\text{cd}} = 90 \text{ MW}$$

comments: $P_{\text{cd}} = 90 \text{ MW}$ for steady operation, with assumed $B_t = 3.5\text{T}$, $I_p = 7.0 \text{ MA}$, $\beta_N = 2.5$ and $f_{\text{cd}} = 0.67$.

2.3 Special challengers for CFETR

2.3.1 Steady state operation

- * Steady state operation is favorable for fusion power plants.
- * However, **191 MW** power is required for non-inductive current **$I_p=10MA$** (see 2.2.4).
- * Assuming the "wall-plug" efficiency to be **0.33**, the required input power for the current drive system will be **573MW**.

Note: "wall-plug" efficiency is the ratio between the output and input power for the non-inductive current drive system.

- * The technology and reliability of the non-inductive current drive system should be significantly improved.

- * Some issues such as the coupling and current drive efficiency in high density operation and possibly with some amount of impurities have to be further studied.**
- * Heating/Current drive system selection?**
- * Calculations and experiments are still required.**

2.3.2 Divertor heat load

- * With 100-200MW power input for current drive, the power load on the divertor plate will be significantly increased**
- * Steady operation further increases the technical requirements.**

2.3.3 Tritium self-sufficiency for the DT fuel cycle

- * The potential of achieving tritium self-sufficiency depends on many system physics and technology parameters.**
- * Compared with DEMO with >1GW fusion power, 100MW fusion power for CFETR are much lower, and the required TBR could be much higher.**

- * Calculations are required to show the consistency:**
To identify physics and technology options and parameters that have large effects on attaining a realistic “window” (ranges of plasma and technology parameters) for tritium self-sufficiency.

2.4 Suggestions

2.4.1 Long pulse rather than steady operation?

(single operation time > 2 hours)

- * **If long pulse operation**, one has to increase the central solenoid size for larger volt-seconds.
- * This choice is simple but increase the machine construction cost.
- * However, there are advantages:
 - Lower operation cost.
 - Lower divertor heat load.
 - Higher operation reliability.

2.4.2 Steady operation only for $I_p=7\text{MA}$ or lower rather than 10MA.

- * For $I_p=7\text{MA}$, **76-90 MW** non-inductive current drive power is still required for steady state operation (see **2.2.5 - 2.2.7**).

2.4.3 Explore the hybrid (improved H-mode) or advanced mode.

- * Increase the bootstrap current fraction for reducing the required current drive power.**
- * but this choice contradicts design principle:
based on standard H-mode.**
- * Studies on the Hybrid (improved H-mode) are still going on.
Further studies are still required.**
- * Maybe we should not exclude the hybrid (improved H-mode) or advanced mode.**
- * High power current drive increases the possibility to obtain and sustain the Hybrid (improved H-mode) or advanced mode.**

3. Summary

- 1. Will the design be based on **long pulse or steady operation?****
- 2. Calculations are required for optimizing machine parameters.**
- 3. Shall we have more than one option in this phase and make a choice later?**
- 4. Special attention is required on the “parameter window” for tritium self-sufficiency.**

Thank you for your attention