

National Research Center "Kurchatov Institute"



Fusion for Neutrons as a Necessary Step to Commercial Fusion

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Fusion for Neutrons - F4N - new formula

- Renewal of interest to fusion neutrons
- Growing interest to steady state operations, which are urgently needed for applications of fusion neutrons
 Several concepts of fusion neutron facilities have been
- Several concepts of fusion neutron facilities have been formulated (FDF, FNS, FDS, Fusion–Fission Hybrids)

WHAT IS the REASON?

Fusion for Energy - **F4E** has a threshold of power amplification Q > 30



F4N has no efficiency limit, Q << 1 is also needed

FNS domain

100

Demonstration Reactor (DEMO) Electric Power Generation ~1000 MWe



Q = $30 \sim 50$ STEADY STATE $\beta_N \sim 3.5 \sim 5.5$

IGNITOR

New products needed to accelerate Fusion

Fast track to Fusion for Energy is defined: ITER ~2020 DEMO ~2035

FPP ~2050



Fast track to Non-Energy Fusion Applications requires

- something that is more valuable than Energy
- something that explores unique features of fusion

Useful Products of DT & DD fusion reactions :

 DT
 DD

 • energy, MeV
 17.6
 3.6
 not much!!

 • neutrons
 1
 0.5
 plenty!!

 • nuclides
 no
 0.5T+0.5 ³He

• Fusion creates more free neutrons (per energy released) than fission by a factor of 20

What is bad for energy production is good for neutron production

The current trend:

«Neutrons (not energy!) may become the first product of fusion»

Fusion device for neutron scattering



... is also a nice device for production of Nobel Prize Winners

Fusion Device is needed for National Health Security

Fusion transmutation of waste is at an immature point in technological readiness, and likely such a unit would be more expensive than a FR. However, due to the better support ratio and the elimination of fuel fabrication, there could be some overall cost benefit to the In-Zinerator concept.



Nuclear Waste (transuraniums) incineration service

Device for benign Energy source



Supporting the high growth rate scenario of Nuclear Industry development by nuclear fuel breeding and waste handling

Fission Energy needs Fusion neutrons

Scales for orientation in the problem

Fission reactor with heat power 3 GW operates at

1020fissions per second~3 x 1020prompt neutrons per second1018delayed neutrons per second





10¹⁸ useful neutrons per second...

is the Everest!



Institute Laue-Langevin 57 MW research reactor

3 MW of DT fusion produce 10¹⁸ useful neutrons per second

Most powerful neutron sources in the world (* - projects)						
NS Type	Facility (location), used nuclides	Deposited Power, MW SS (Peak)	Rate, 10 ¹⁷ n/s SS (Peak)	Neutron Power Output, MW SS (Peak)	Max. Neutron Flux Density, n/cm ² s	
1. Fission reactors	ILL (Grenoble, France), U ²³⁵	56	10	1.5	1015	
	PIK (Gatchina, Russia), U ²³⁵	100	20	3	4.5×10 ¹⁵	
	IBR-2 (Dubna, Russia), Pu ²³⁹	2 (1500)	0.6 (500)	0.03 (25)	10 ¹⁶	
2. Accelerators	SNS (ORNL), p, Hg	1 (30000)	1 (30000)	0.3 (10000)	10 ¹⁶	
	LANSCE (LLNL), p, W, Pb, Bi	0.1 (10000)	0.1 (10000)	0.03 (3000)	10 ¹⁶	
	*IFMIF (being negotiated), D, Li	9	1	1	10 ¹⁵	
3. Tokamaks	JET (Abingdon, UK), D, T	0 (16)	0 (60)	0 (13)	10 ¹³	
	*JT-60SA (Naka, Japan), D	0.01 (0.5)	0.01 (2)	0 (0.4)	1011	
	*ITER (Cadarache, France), D, T	500	1800	400	4×10 ¹³	
4. Stellarators	LHD (Toki, Japan), D	20	*0.2	0.002	10 ¹⁰	
5. Muon catalysis	*LAMPF (LLNL), p, Hg, D, T	1	*1.8	1.4	10 ¹²	
6. Z-pinch	*Z (Albuquerque), D, T	30	70	24	1017	
7. Laser system	*LIFE (LLNL), D, T	1000	2100	800	1017	

Beam-plasma interaction is a good source of neutrons

 Optimal fusion energy and neutron production are realized at different plasma conditions:

Beam-plasma fusion rate has maximum at lower $n\tau_E \sim 10^{13}$ cm⁻³ s



Dependence of P_f and Q_f on $n_e \tau_E$ for an 8-keV D-T plasma heated by 200-keV D beams. Maximum Q_f is attained for the plasma composition given by n_T/n_i , where n_i is the bulk-ion density. $p=0.655 \text{ J/cm}^3$. Radiation loss is included in τ_E . Alpha-particle effects are neglected.

Challenges for "Fusion for Neutrons"

- Free neutrons
- Transmutation
- Fuel breeding

Compact tokamaks with copper coils and MW fusion power may compete with contemporary neutron sources (fission reactors and spallation neutron sources)

Steady State Operations in neutron environment is the basic requirement for FNS for hybrids

Pulsed Operation acceptable for research



Interest to hybrid systems in Russia came back because:

Resources of U-235 for development of power generation by thermal reactors are limited

•Fast reactors as the basis for future large-scale nuclear industry with acceptable economy are at a stage of concept selection and still have unclear prospects due to technological difficulties

A part of Russian fission community considers fusionfission hybrid systems based on fusion neutron source (FNS) as a possible solution of the fission power problems



Schematic diagram of a Fusion-Fission Hybrid





Time scale of hybrids development

Current activity:

Development the physical prototype T-15 and demonstration FNS for hybrid systems

Several options for demonstration tokamak- FNS with warm and superconducting electromagnetic system have been considered to ensure the maintenance of steady-state mode at conceptual level since 2009

Plans:

Facilities to be constructed and put into operation:Physical prototype of FNS (T-15, Globus MU)2013TIN-1 (demonstration tokamak-FNS)2018Pilot plant2022Industrial hybrid reactor2030

Basic goals of FNS-ST design

- To formulate a concept of a MW range fusion neutron source as small in size as possible
- To optimize beam-plasma fusion regime
- To determine the device SS Operation domains
- To define key physical issues of the device
- To outline the R&D needed to support the project

FNS-ST project

• The mission of the FNS-ST is to be a prototype of a fusionfission hybrid system with the fusion power below 10 MW and fission power below 100 MW, being capable to produce neutrons with fast/thermal spectra with the plant life time of several decades.

Main design constraints:

- Minimization of the device size is desirable to reach highest neutron flux density

- Reducing the total electric power consumption below 50 MW follows from the desire to keep the capital cost <200 M\$ and operation cost <30 M\$ with the duty factor of 0.3

- Tritium consumption to be less than 100 grams per year

B.V. Kuteev et. Al. "Plasma Physics Reports, 2010, vol.36, pp.281

FNS-ST: basic parameters and cut-view

R, m	0.5
R/a	1.67
k	2.75
δ	0.5
I _p , MA	1.5
В _т , Т	1.5
n, 10 ²⁰ m ⁻³	1
P _{wall} , MW/m ²	0.2
E _b , keV	130
P _b , MW	10
Angle NBI, deg	30
P _{EC} , MW	5
H-factor	1-2
β _N	5
f _{non-ind}	1.0
P _{diss} , TF, MW	14
P _{diss} , PF, MW	6.0
S _{wall} , m2	13
V _{pl} , m3	2.5



$$S = \frac{I_p}{aB_T} q_{95} \sim \frac{1}{A} \left[1 + k^2 (1 + 2\delta^2) \right] \sim 30$$

B. V. Kuteev et al. Nuclear Fusion 51 (2011) 073013

Wall and divertor thermal loads



- P_{heat}/R and P_{heat}/S determine divertor plate and wall loads correspondingly
- P P_{heat}/R is of the order of ITER values, therefore ITER divertor technologies may be used
- P_{heat}/S is very high exceeding ITER values by factor of 3, therefore ITER divertor technologies should be used for the first wall as well

• In basic FNS-ST regime P_{heat}/S is close to Alcator C-MOD values

FNS-ST divertor concept



Main divertor components are:

- Magnetic coils
- Divertor cassettes; W tiles protected by lithium films
- Lithium dust injectors and collectors for plasma-wall interaction control

B. V. Kuteev et al. Nuclear Fusion 50 (2010) 075001

Pumping and gas supply systems

Local gas puffing for detachment (or partial detachment)

V.Yu. Sergeev et al. Plasma Physics Reports, 2011 TBP

Magnet system with twisted central pole



The toroidal magnet system: Cu alloy, 20 tons, 80 m length, 250 kA current, 46.8 V, 11.7 MW, $T_{max} = 142^{\circ}$ C (a) with twisted central pole (b).

The vacuum vessel assembly is inside the TF magnet (c).

Model used for FNS-ST neutronics



- Source strength 10¹⁸ n/s (3 MW) provides thermal neutron flux 5 10¹⁴ n/cm²s
- Be central post allows to reach ~10¹⁵ n/cm²s

Two nuclear fuel cycles are considered



U-Pu 1Pu+1T per 1n(DT)

Th-U 0.6U+1T per 1n(DT)

Conclusions

- Development of "Fusion for neutrons" paradigm gives copper coil compact tokamak a chance to become a feasible prototype for commercial applications
- Demonstration FNS prototype urgently needs SSO technologies
- MW range FNS is sufficient to demonstrate new neutron and hybrid technologies
- FNS needs further research on specific issues of anisotropy plasma physics and advanced fusion technologies and materials
- More concepts of neutron applications are needed to advance fusion development

-> Conceptual design stage of FNS-ST in 2011-2012

Demand for E&N from Fusion



ITER -> DEMO -> PROTO

pure fusion

Q = 5 -> ~30 -> >30

F4Neutrons FNS-P -> FNS-D -> FNS-C -> FNS-T -> FNS-F hybrids plasma demo control transmutation fuel -> 10¹⁹ -> > 10²⁰ n/s N-Rate = 0-> ~**10**¹⁸ pure and F4Research FNS-ST **FNS-R** -> hybrids **10**¹⁶ N- thermal flux = $\sim 10^{15}$ n/cm²s -> **10**¹⁷ n/cm²s -> Ignitor

Neutron loading and production rate

